

Neutron Transport Theory Lecture Note (3)
-Neutron Transport Equation -

Toru Obara
Tokyo Institute of Technology

3. Neutron Transport Equation

3.1 Neutron Density and Flux

Definition

Angular neutron density $n(\mathbf{r}, E, \hat{\Omega}, t) d^3r dE d\hat{\Omega}$

$$\equiv \left[\begin{array}{l} \text{Expected number of neutrons in } d^3\mathbf{r} \text{ about } \mathbf{r}, \\ \text{energy } dE \text{ about } E, \text{ moving in direction } \hat{\Omega} \\ \text{in solid angle } d\hat{\Omega} \text{ at time } t \end{array} \right]$$

Angular neutron flux

$$\varphi(\mathbf{r}, E, \hat{\Omega}, t) \equiv v n(\mathbf{r}, E, \hat{\Omega}, t)$$

where v is the neutron speed

$$v = \sqrt{\frac{2E}{m}}$$

Angular current density

$$\mathbf{j}(\mathbf{r}, E, \hat{\Omega}, t) \equiv v \hat{\Omega} n(\mathbf{r}, E, \hat{\Omega}, t) = \hat{\Omega} \varphi(\mathbf{r}, E, \hat{\Omega}, t)$$

Neutron density

$$N(\mathbf{r}, E, t) = \int_{4\pi} d\hat{\Omega} n(\mathbf{r}, E, \hat{\Omega}, t)$$

and

$$N(\mathbf{r}, t) = \int_0^\infty dE N(\mathbf{r}, E, t) = \int_0^\infty dE \int_{4\pi} d\hat{\Omega} n(\mathbf{r}, E, \hat{\Omega}, t)$$

Neutron flux

$$\phi(\mathbf{r}, E, t) = \int_{4\pi} d\hat{\Omega} \varphi(\mathbf{r}, E, \hat{\Omega}, t)$$

and

$$\phi(\mathbf{r}, t) = \int_0^\infty dE \phi(\mathbf{r}, E, t) = \int_0^\infty dE \int_{4\pi} d\hat{\Omega} \varphi(\mathbf{r}, E, \hat{\Omega}, t)$$

unit $[\text{cm}^{-2} \cdot \text{s}^{-1}]$

Neutron current density

$$\mathbf{J}(\mathbf{r}, E, t) \equiv \int_{4\pi} d\hat{\Omega} \mathbf{j}(\mathbf{r}, E, \hat{\Omega}, t)$$

and

$$J(\mathbf{r}, t) \equiv \int_0^\infty dE J(\mathbf{r}, E, t) = \int_0^\infty dE \int_{4\pi} d\hat{\Omega} \mathbf{j}(\mathbf{r}, E, \hat{\Omega}, t)$$

3.2 Neutron Transport Equation

Consider arbitrary volume V . The number of neutrons in V with energy E in dE and traveling in a direction $\hat{\Omega}$ in $d\hat{\Omega}$ within this volume is,

$$\left[\int_V n(\mathbf{r}, E, \hat{\Omega}, t) d^3r \right] dE d\hat{\Omega} .$$

The time rate of change of this number is given by a balance relation.

$$\frac{\partial}{\partial t} \left[\int_V n(\mathbf{r}, E, \hat{\Omega}, t) d^3r \right] dE d\hat{\Omega} = (\text{gain in } V) - (\text{loss from } V)$$

If we assume that the arbitrary volume is chosen not to depend on time,

$$\frac{\partial}{\partial t} \left[\int_V n(\mathbf{r}, E, \hat{\Omega}, t) d^3r \right] dE d\hat{\Omega} = \left[\int_V \frac{\partial n}{\partial t} d^3r \right] dE d\hat{\Omega}$$

Gain mechanisms :

- (1) Any neutron source in V (e.g. fission)
- (2) Neutrons streaming into V through the surface of the V
- (3) Neutrons of different E' , $\hat{\Omega}'$ suffering a scattering collision in V that changes E' , $\hat{\Omega}'$ into the E , $\hat{\Omega}$ of interest

Loss mechanism :

- (4) Neutrons leaking out through the surface of the V
- (5) Neutrons suffering a collision (absorption and scattering)

Neutron balance relationship in a unit volume

(Neutron transport equation)

$$\begin{aligned} \frac{\partial n}{\partial t} + \mathbf{v}\hat{\Omega} \cdot \nabla n + \mathbf{v}\Sigma_t n(\mathbf{r}, E, \hat{\Omega}, t) \\ = \int_{4\pi} d\hat{\Omega}' \int_0^\infty dE' v' \Sigma_s(E' \rightarrow E, \hat{\Omega}' \rightarrow \hat{\Omega}) n(\mathbf{r}, E', \hat{\Omega}', t) + S(\mathbf{r}, E, \hat{\Omega}, t) \quad (1) \end{aligned}$$

where,

$$\Sigma_s(E' \rightarrow E, \hat{\Omega}' \rightarrow \hat{\Omega}) \equiv N \sigma_s(E' \rightarrow E, \hat{\Omega}' \rightarrow \hat{\Omega})$$

$\sigma_s(E' \rightarrow E, \hat{\Omega}' \rightarrow \hat{\Omega})$: Cross section that characterize scattering from an incident energy E' , direction $\hat{\Omega}'$ to find energy E and direction $\hat{\Omega}$.
(double differential scattering cross section)

$$S(\mathbf{r}, E, \hat{\Omega}, t) d^3r dE d\hat{\Omega} \equiv \left[\begin{array}{c} \text{rate of source neutrons appearing} \\ \text{in } d^3r \text{ about } \mathbf{r}, dE \text{ about } E, \text{ and } d\hat{\Omega} \\ \text{about } \hat{\Omega} \end{array} \right]$$

(neutron source)

3.3 Simplifications to the Neutron Transport Equation

- The one-speed approximation : Assuming that one can characterize the neutrons by a single energy or speed.

One-speed neutron transport equation

$$\frac{1}{v} \frac{\partial \varphi}{\partial t} + \hat{\Omega} \cdot \nabla \varphi + \Sigma_t \varphi(\mathbf{r}, \hat{\Omega}, t) = \int_{4\pi} d\hat{\Omega}' \Sigma_s(\hat{\Omega}' \rightarrow \hat{\Omega}) \varphi(\mathbf{r}, \hat{\Omega}', t) + S(\mathbf{r}, \hat{\Omega}, t)$$

- Isotropic neutron sources

$$S(\mathbf{r}, \hat{\Omega}, t) = \frac{1}{4\pi} S(\mathbf{r}, t)$$

- Isotropic scattering

$$\Sigma_s(\hat{\Omega}' \rightarrow \hat{\Omega}) = \frac{1}{4\pi} \Sigma_s$$

The one-speed neutron transport equation with isotropic scattering and sources

$$\frac{1}{v} \frac{\partial \varphi}{\partial t} + \hat{\Omega} \cdot \nabla \varphi + \Sigma_t \varphi(\mathbf{r}, \hat{\Omega}, t) = \frac{\Sigma_s}{4\pi} \int_{4\pi} d\hat{\Omega}' \varphi(\mathbf{r}, \hat{\Omega}', t) + \frac{S(\mathbf{r}, t)}{4\pi}$$

In a steady state

$$\hat{\Omega} \cdot \nabla \varphi + \Sigma_t \varphi(\mathbf{r}, \hat{\Omega}) = \frac{\Sigma_s}{4\pi} \int_{4\pi} d\hat{\Omega}' \varphi(\mathbf{r}, \hat{\Omega}') + \frac{S(\mathbf{r})}{4\pi}$$

In a one-dimensional plane geometry

$$\mu \frac{\partial \varphi}{\partial x} + \Sigma_t \varphi(x, \mu) = \frac{\Sigma_s}{2} \int_{-1}^{+1} d\mu' \varphi(x, \mu') + \frac{S(x)}{2}$$

where

$$\mu = \cos \theta$$